

Task 16

Evaluation of Fluorapatite as a Waste-Form Material

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BACKGROUND

High Temperature Gas-Cooled Reactor (HTGR) designs are currently being designed and evaluated as part of the Advanced Fuel Cycle Initiative, both as a future nuclear reactor design (in the Generation IV reactor program) and as a potential system for burning plutonium in a dual-tier transmutation strategy. HTGR designs use a TRISO-coated fuel (a silicon-carbide and pyrocarbon composite coating) to provide much of the passive containment for radionuclides.

Although this fuel form is quite stable and comprises one of the key components in the safety performance of these reactor systems, TRISO-coated fuel is comparatively difficult to recycle. If HTGR systems are to be used as part of either the first tier of a transmutation strategy or as primary power reactors, the plutonium and other actinides in the used TRISO fuel must be processed to recycle plutonium and permit recovery of minor actinides and other fission products produced.

Argonne National Laboratory has proposed a new extraction procedure to handle TRISO-coated fuels, the Fluoride Extraction Process, or FLEX. The FLEX process is designed to separate the uranium in the fuel from the actinides and most fission products by taking advantage of the unique properties of uranium hexafluoride (UF_6). In the FLEX process, the used TRISO fuel is reacted with zirconium fluoride salt, forming UF_6 and the fluoride salts of the actinides and fission products. At process temperatures, the UF_6 volatilizes into a gas, and is released from the molten salt mixture. This leaves behind the actinides and most fission products in a fluoride salt, which is subsequently processed using pyrochemical techniques to recover the actinides and other long-lived fission products for transmutation. The UF_6 is then cooled, causing it to sublime into solid form, which is processed for disposal or reuse.

The primary waste stream from the FLEX process is the fission products from the fuel, which are in a zirconium fluoride salt at the end of the process. Due to the fluorine in this waste stream, the fluoride salts are unsuitable for conversion into the traditional borosilicate waste glass. Without a suitable disposal form, this process can not be deployed.

This research attempts to develop a waste form for disposing of the zirconium fluoride fission product waste stream. Fluorapatite, a naturally-occurring fluorinated calcium phosphate, has been identified as a potential matrix for the entombment of this waste stream. If the efficacy of fluorapatite-based waste-storage can be demonstrated, then new and potentially more efficient options for handling and separating high-level wastes, based on fluoride-salt extraction, will become feasible.



Green colored crystal (fluorapatite)

RESEARCH OBJECTIVES AND METHODS

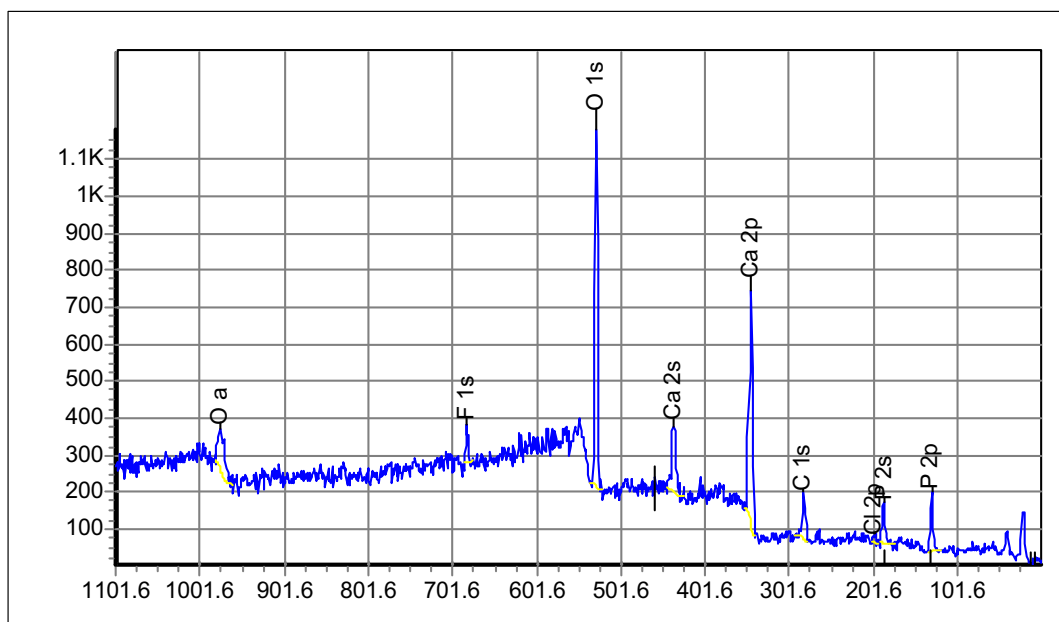
The following are the specific research objectives:

- To develop a waste matrix for the disposal of the fission product waste stream from the FLEX process;
- To develop a process to make synthetic fluorapatite that incorporates the FP-bearing ZrF_4 salt;
- To develop a fundamental understanding of the chemistry of this new waste form in order to better predict its long term behavior in a repository environment;
- To develop a fundamental understanding of natural, fluoride-bearing mineral phases to use as natural analogs to bound the predicted behavior of the FLEX fission product waste stream.

The research effort has been divided along two parallel paths: the Fabrication path, led by collaborators at the Khoplin Radium Institute (KRI) in St. Petersburg, Russia; and the Characterization path, led by researchers from UNLV.

The Fabrication path is focused on examining and evaluating various techniques for fabricating synthetic fluorapatite; synthesizing synthetic fluorapatite; and examining the impacts of waste loading and other fabrication process factors on the performance of the synthetic fluorapatite as a potential waste form.

The Characterization path is focused on adapting and refining the X-ray spectroscopy techniques currently used to characterize borosilicate glass for use in examining the fluorapatite system. This path will also encompass the examination of the ceramic and synthetic mineral waste forms created at KRI, with subsequent examination of these techniques to develop a molecular-level understanding of natural fluorapatite and other fluorine-bearing natural phases as natural analogs for the waste form. These techniques will also be used to examine the changes in surface chemistry caused by environmental degradation of these materials.



Composition Table

XPS Line	Atom %
C 1s	16.494
F 1s	2.914
O 1s	46.569
Ca 2p	19.344
Cl 2p	1.000
P 2p	13.679

X-ray Photoelectron Spectroscopy (XPS) data for Hydroxyapatite.

RESEARCH ACCOMPLISHMENTS

Research highlights for the first year can be summarized as follows:

- Baseline XPS, FT-IR, and Raman spectra have been obtained for hydroxyapatite and fluorapatite.
- Natural fluorapatite crystals have been obtained commercially, and will be examined spectroscopically to determine what contaminants naturally occur.
- Plans were developed for chemically preparing samples in which some of the calcium in apatite materials is substituted by non-radioactive actinide surrogates or elements produced by decay of actinides.
- SEM images of natural fluorapatite crystals have been obtained, indicating the presence of natural inclusion containing transition metals (e.g. Ni), possibly providing useful information for comparison of man-made wastes with the same or similar species.

FUTURE WORK

Continued research efforts will focus on developing a fluorapatite-based waste form and a fabrication process/flow sheet for the FLEX process fission product waste stream. The research will include characterizing the molecular structure and corrosion behavior of natural fluorapatites and fluorapatite-based waste forms.

Waste form development at KRI involves the formulation, synthesis, and examination of ceramic samples to investigate the impact of processing parameters and composition on material properties (e.g. homogeneity) and performance (via leach testing). The most promising fabrication process developed will be used to synthesize the ceramic in varying compositions to examine the impact of process parameters, ceramic formulation, and waste loading on the final ceramic phase. Based on these experiments, a baseline composition and fabrication process will be established. Finally, alternate fabrication processes, compositions, and potential alternate waste matrices will be examined and compared against the baseline composition.

Waste form characterization carried out at UNLV employs state of the art techniques that characterize the molecular structure of both natural fluorapatite and fluoride-bearing minerals and the fluorapatite-based ceramic waste form. Changes in the surface/interfacial chemistry of these materials as they undergo reactions with species in the environment will also be examined to help develop a basis for understanding the corrosion chemistry that the waste form and its natural analogs may experience under repository conditions.

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